

#### **Development and testing**

#### of High Heat Flux Components for ITER

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## **Talk outline**

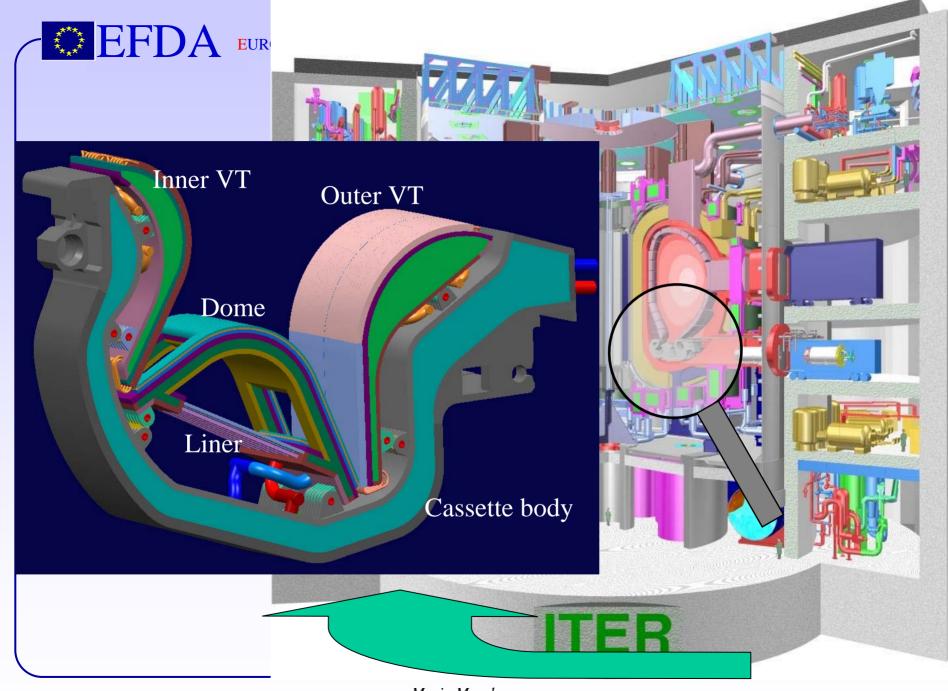
- The ITER Divertor
- Reference geometry for the High Heat Flux Components
- Technology development
- Non-destructive testing
- Acceptance criteria
- Present and future plans



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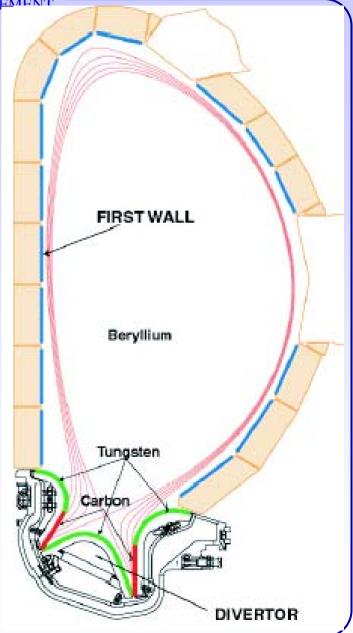
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#### The main function of the divertor system is to exhaust the major part of the alpha particle power as well as helium and impurities from the plasma

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# Beryllium

- Low atomic number
- Oxygen gettering capability
- Absence of chemical sputtering
- High thermal conductivity

#### CFC

- Longest lifetime
- Absence of melting
- Excellent thermal shock resistance
- Very high thermal conductivity
- Low atomic number



#### **CFC Material**

#### **Thermal Conductivity at RT (W/mK)**

Direction	Х	Y	Ζ
NB31	323	117	115
Concept 2	340	113	78

#### **Tensile Strength at RT (MPa)**

NB31	130	30	19
Concept 2	113	35	8

## Tungsten

Lowest sputtering

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- Highest melting point
- High thermal conductivity
- No concerns over tritium inventory

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HIGH HEAT FLUX COMPONENTS	FOSSILE FIRED BOILER WALL (ABB)	FISSION REACTOR (PWR) CORE	ITER DIVERTOR		
DESIGN					
HEAT FLUX					
<ul> <li>average MW/m<sup>2</sup></li> <li>maximum MW/m<sup>2</sup></li> </ul>	0.2 0.3	0.7 1.5	$3-5 \\ 10-20$		
Max heat load MJ/m <sup>2</sup>	-	-	10		
Lifetime years	25	4	3		
Nr. of full load cycles	8000	10	3000		
Neutron damage dpa	-	10	0.2		
Structure material	Ferritic-Martens. steel	Zircaloy - 4	CuCrZr & CFC/W		
<u>Coolant</u> - pressure MPa	Water-Steam 28	Water 15	Water 4		
- temperature °C	280-600	285-325	100 - 150		
- velocity m/s - leak rate g/s	3 <50	5 <50(SG)	9 - 11 <10 <sup>-7</sup>		



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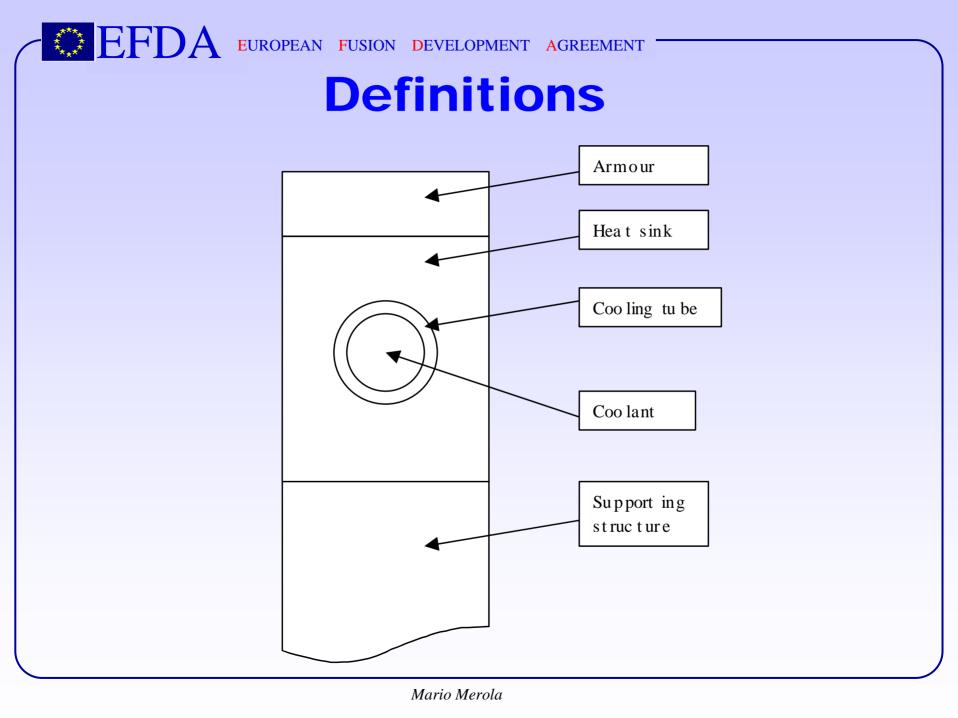
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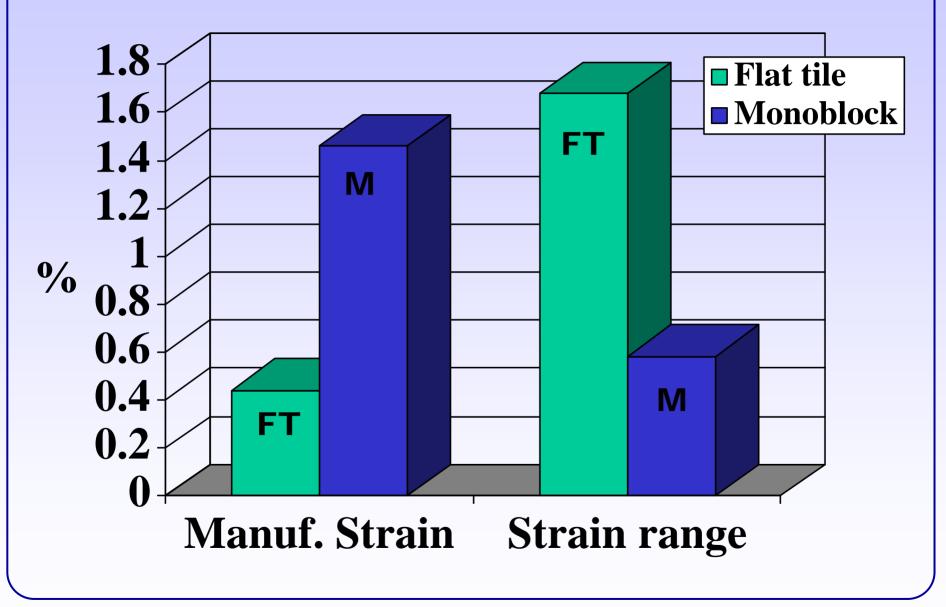
# **Possible PFC geometries**

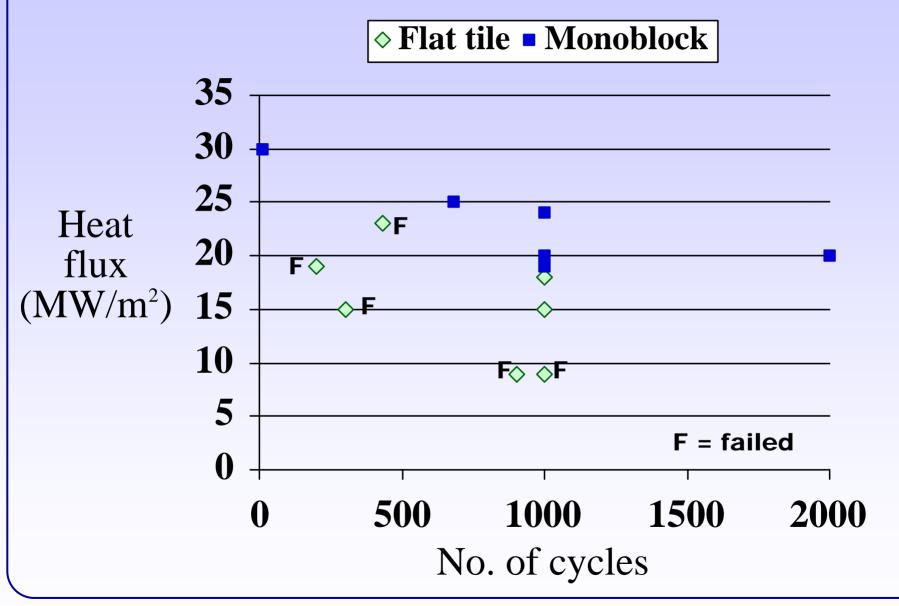




#### Monoblock

Flat tile





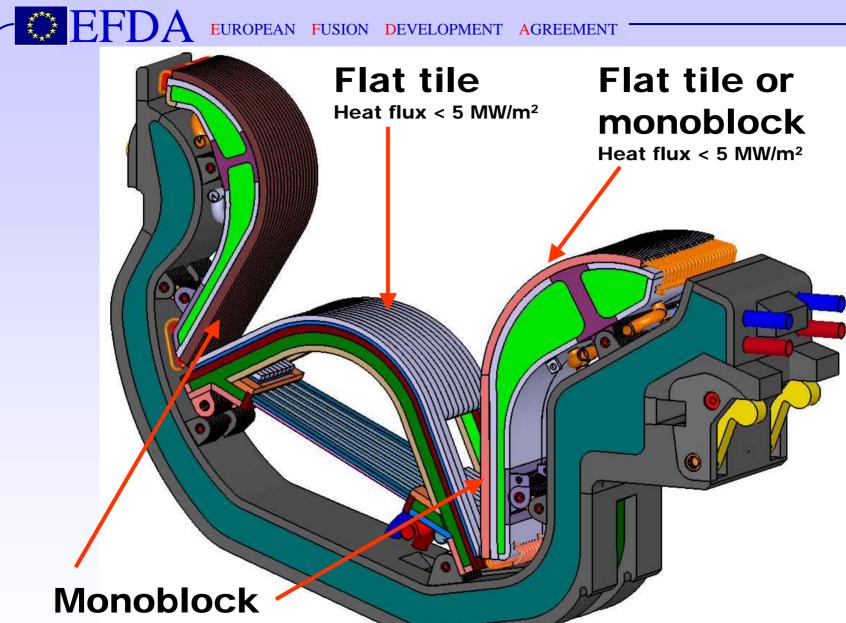
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## Conclusion

Manufacturing of flat tile components is easier than monoblocks

Thermal fatigue performances of monoblock components are superior than those of flat tiles



Heat flux up to 20 MW/m<sup>2</sup>



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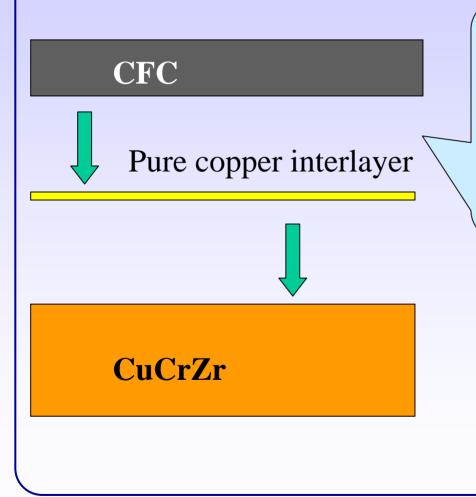
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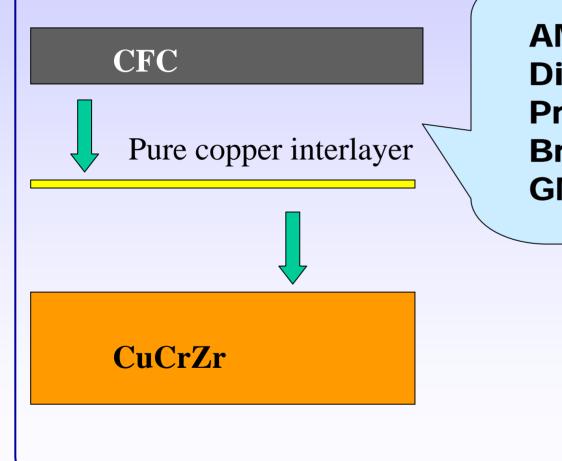
## Armour to heat sink joints



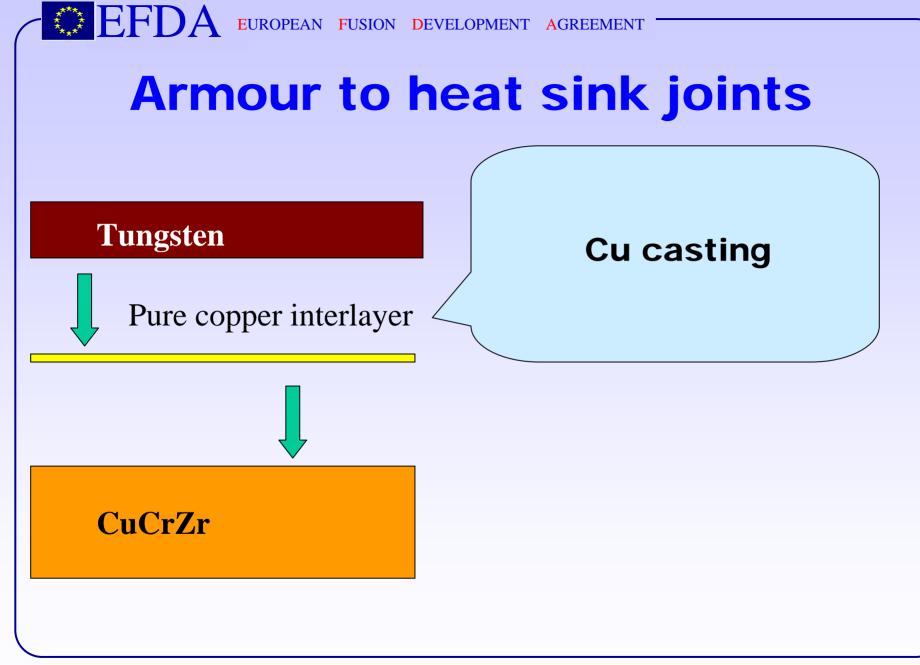
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Copper does not wet carbon Wetting agents may lead to the formation of brittle intermetallics or compounds with a low melting point Large thermal expansion mismatch

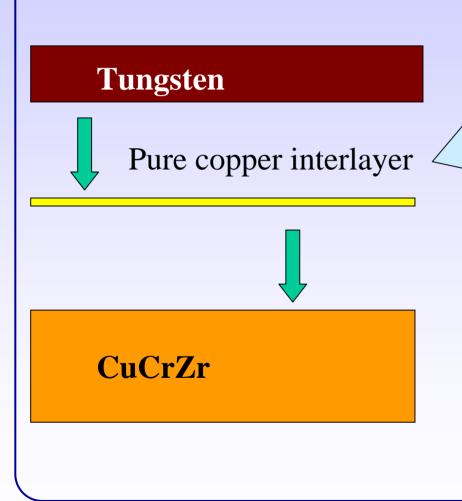
## Armour to heat sink joints



AMC Direct casting Pre-brazed casting Brazing GMP



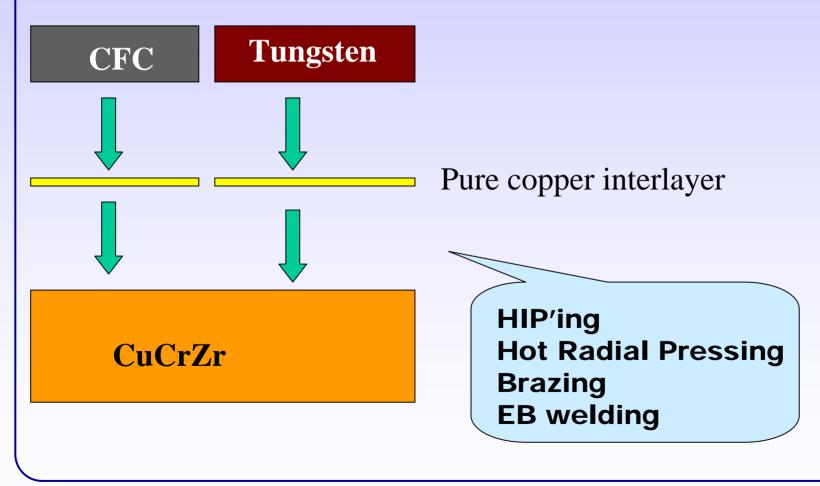
## Armour to heat sink joints



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Casting Cu onto W requires adequate experience to ensure a good wetting of W and to prevent the formation of bubble in the cast Cu

## Armour to heat sink joints



EFDA EUROPEAN FUSION DEVELOPMENT AGREEMENT CuCrZr: a "difficult" material

It reaches an optimum in strength after a thermomechanical treatment involving:

- 1) first a solution annealing at high temperature (>980 C) to dissolve the alloying elements (Cr, Zr)
- 2) then a water quench to keep the alloying elements in supersaturated solid solution at room temperature
- 3) finally an ageing treatment at intermediate temperatures (475 C, 3 hrs) to decompose the supersaturated solid solution into a fine distribution of precipitates.

## CuCrZr: a "difficult" material

The manufacturing route shall be carefully defined.

Thermal excursions above the ageing temperature can overage the alloy with a significant decrease of strength.

Thermal excursions above the solution annealing temperature can lead to a significant decrease of the thermal conductivity, due to the dissolution of precipitates.

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#### First manufacturing strategy

- Starting point: CuCrZr in the SA status
- The ageing heat treatment is performed at the same time of the HIP joining process

# What is the maximum allowable HIP cycle in terms of temperature and time ?

## Allowable manufacturing HIP cycle

 Any "ageing" heat treatment below 600 °C gives acceptable mechanical strength

## Second manufacturing strategy

- Starting from CuCrZr in any status, the SA heat treatment is performed at the same time of the brazing joining process
- Then the component is "fast cooled"
- Then the ageing heat treatment is applied

#### What is the required cooling rate to ensure an adequate recovery of the thermal and mechanical properties ?

### Allowable manufacturing brazing cycle

 Any "fast cooling" after brazing with a rate > 1 C/s enables an acceptable recovery of the mechanical strength EFDA EUROPEAN FUSION DEVELO

Vertical Target Medium-Scale Prototype

## **Test results**

- W macrobrush: 15 MW/m<sup>2</sup> x 1000 cycles
- CFC monoblock
   20 MW/m<sup>2</sup> x 2000 cycles
- CHF test > 30 MW/m<sup>2</sup>

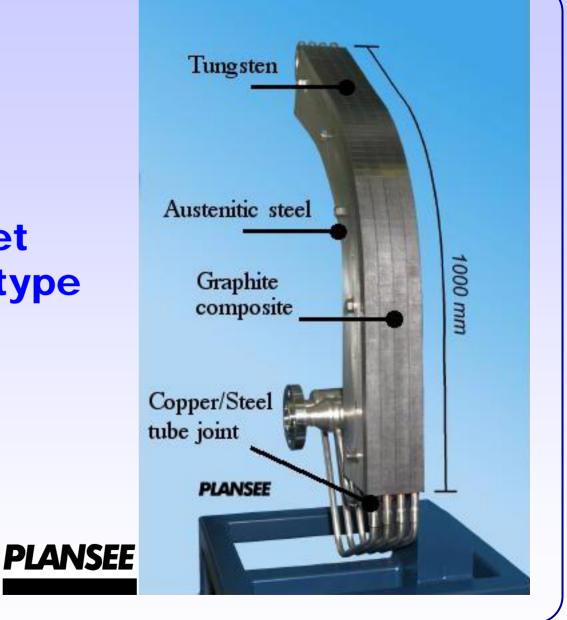
#### PLANSEE





#### Vertical Target Full-Scale Prototype

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#### Vertical Target Full-Scale Prototype



- W monoblocks:
   10 MW/m<sup>2</sup> x 1000 cycles
- CFC monoblock

   MW/m<sup>2</sup> x 1000 cycles
   MW/m<sup>2</sup> x 1000 cycles
   MW/m<sup>2</sup> x 1000 cycles



#### Vertical Target components with W armour

#### 10 MW/m<sup>2</sup> x 1000 cycles 16 MW/m<sup>2</sup> x 1000 cycles 21 MW/m<sup>2</sup> x 1000 cycles





Vertical Target component with W armour



Ansaldo Ricerche Srl

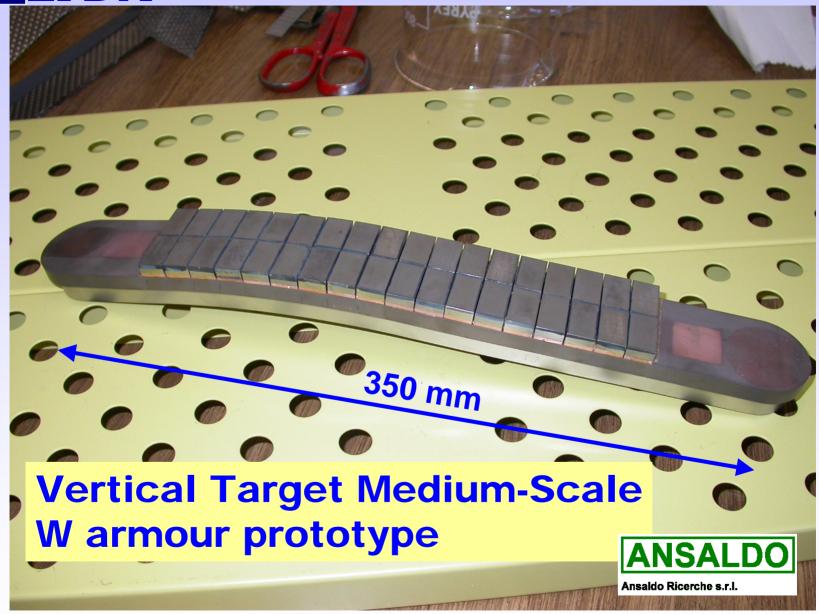
Small Vertical-Target Divertor mock-up - 300 mm Tungsten monoblock -

12-mm thick Tungsten tiles joined to an heat-treated Copper-Chromium-Zirconium alloy pipe BEFORE HIGH HEAT FLUX TEST

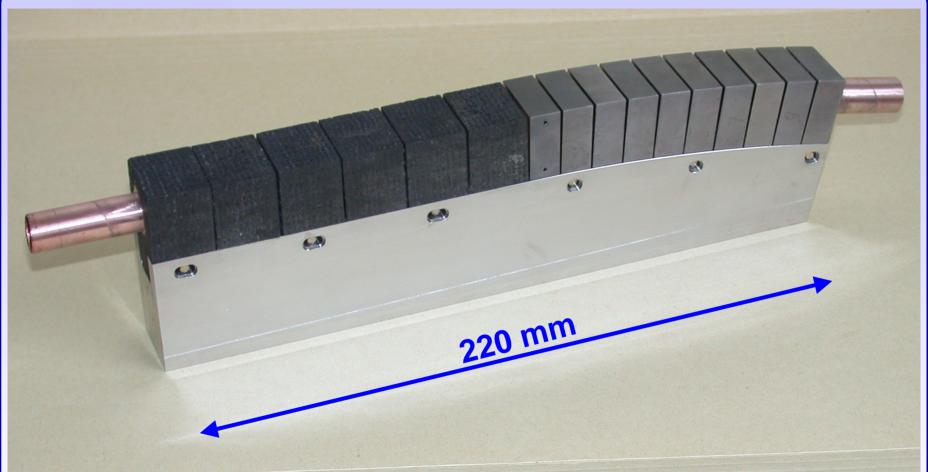
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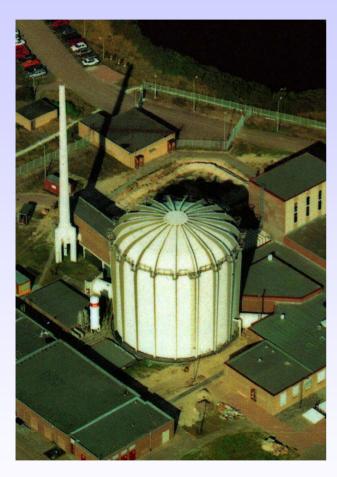
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### Vertical Target Medium-Scale Prototype



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High Flux Reactor Petten, Netherlands

PARIDE 1:

- temperature: 350°C
  - target fluence: 0.5 dpa

#### PARIDE 2:

- temperature: 700°C
  - target fluence: 0.5 dpa

#### PARIDE 3:

- temperature: 200°C
- target fluence: 0.2 dpa

#### PARIDE 4:

- temperature: 200°C
  - target fluence: 1 dpa



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#### **Testing of Tungsten Mock-Ups**

#### Unirradiated

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- 1000 cycles x 14 MW/m<sup>2</sup> – no failure

#### 200°C, 0.1 and 0.5 dpa in tungsten

- Failure limit: 10 MW/m<sup>2</sup>

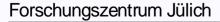




#### Unirradiated - 1000 cycles x 20 MW/m<sup>2</sup> – no failure

#### 200°C, 0.1 and 0.5 dpa in tungsten

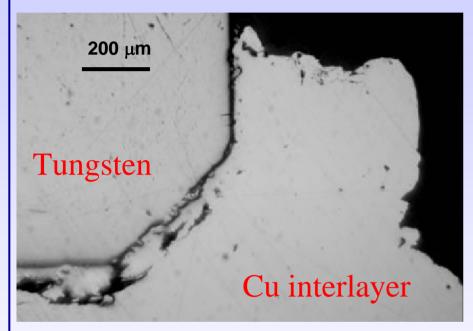
- Successfully tested up to 18 MW/m<sup>2</sup>



in der Helmholzgemeinschaft EURATOM-Association

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#### Thermal fatigue testing of a tungsten macrobrush mock-up irradiated in the PARIDE 3 experiment



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Irradiation condition:  $200^{\circ}C - 0.1 \text{ dpa}$  (in W)

High heat flux test: 1000 cycles at 10 MW/m<sup>2</sup>



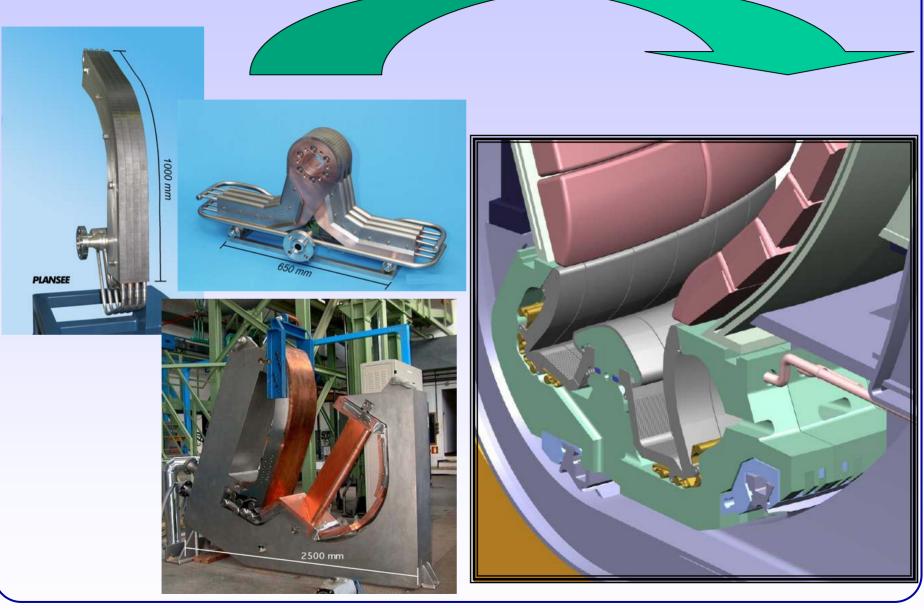


### Irradiation of W armoured mock-ups

The irradiated pure Cu interlayer leads to a reduction of the high heat flux performances in a flat tile geometry.

The monoblock solution seems not to be affected by this problem.

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## **Non-Destructive Testing**

- Infrared thermography
- Ultrasonic inspections

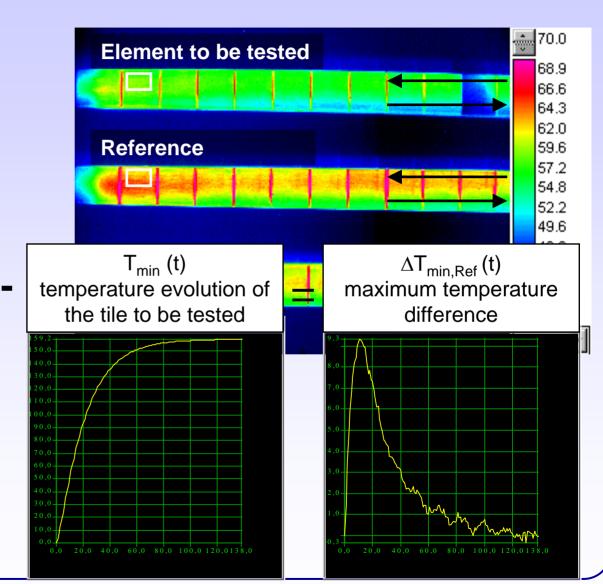
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#### Transient Thermography Inspection

 $T_{min,Ref}(t)$  temperature evolution of

the reference tile

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**Flow direction** 

## **Non-Destructive Testing**

- Infrared thermography proved to be an essential inspection technique for the CFC/Cu joint
- Ultrasonic inspections have also been successfully applied on the CFC/Cu joint but their effectiveness of depends on the manufacturing technologies
- Ultrasonic inspections is the preferred method to inspect metallic joints (W/Cu and Cu/CuCrZr)



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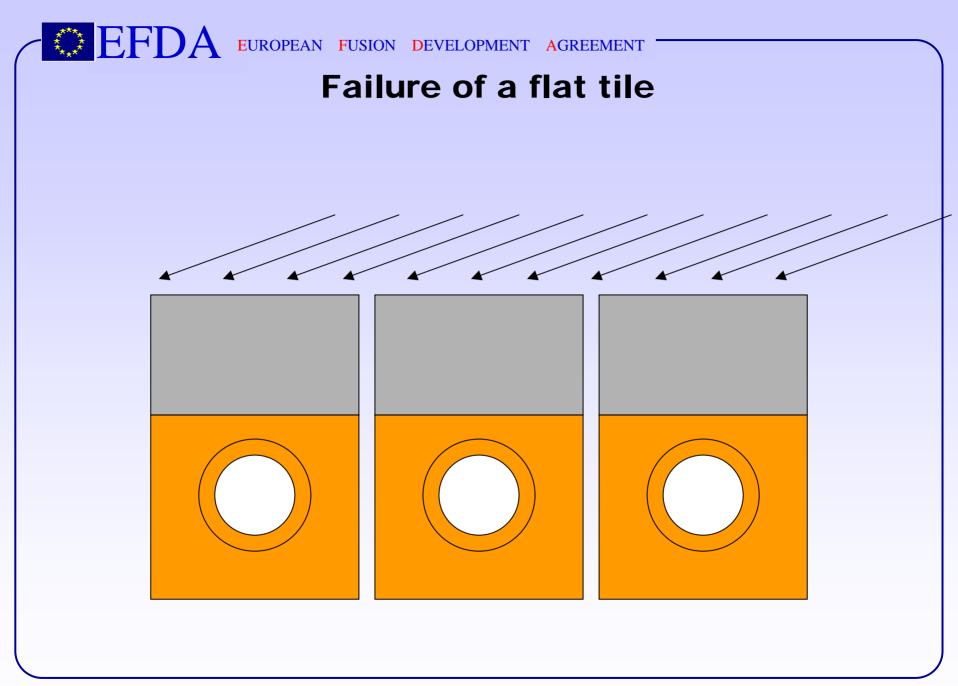


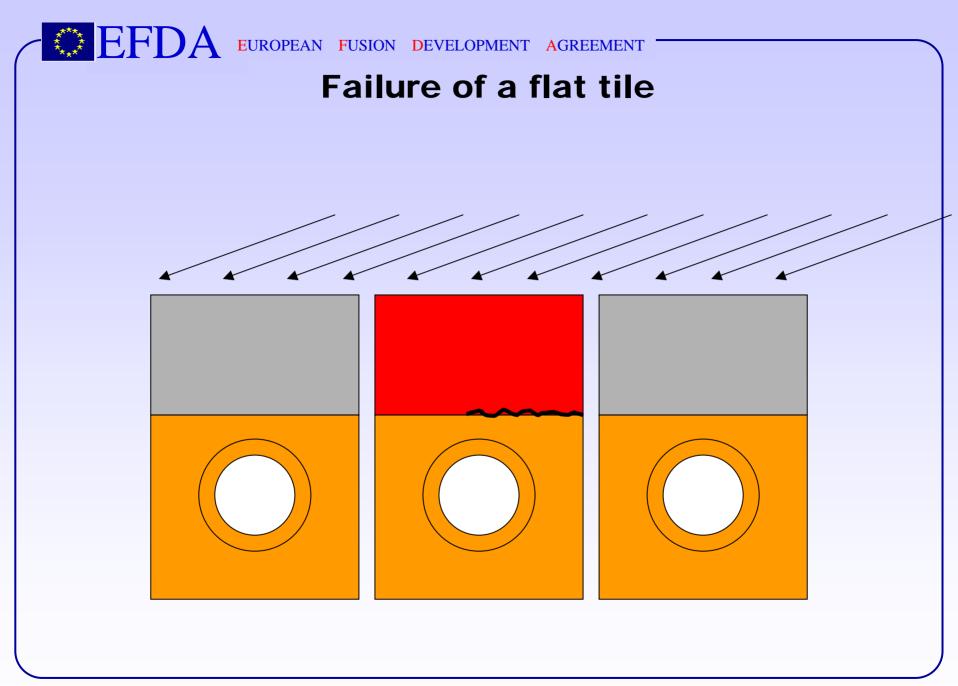
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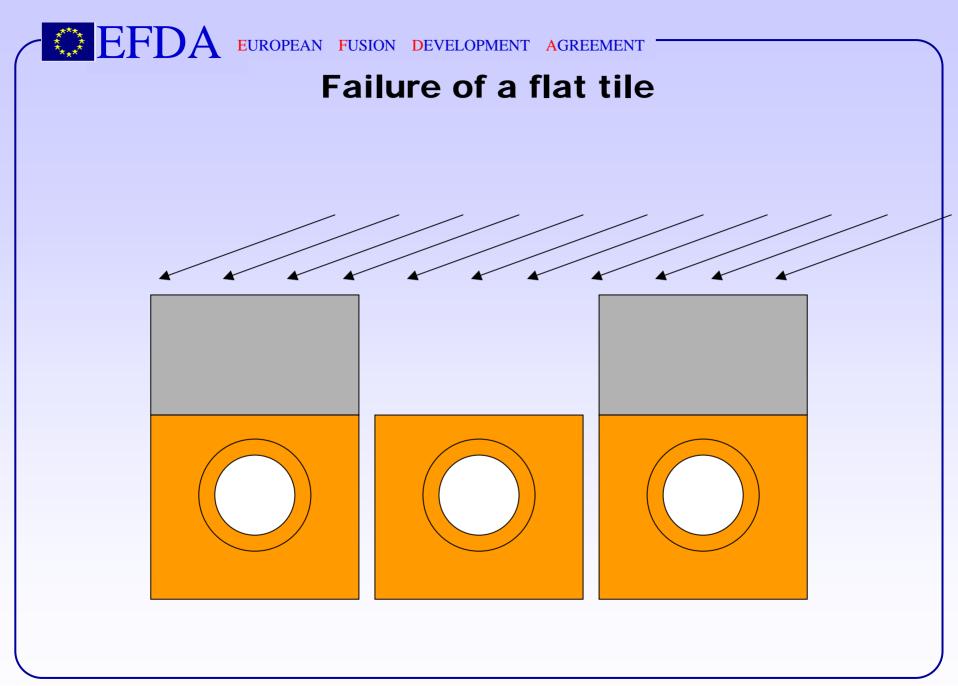
# Scope of the activity

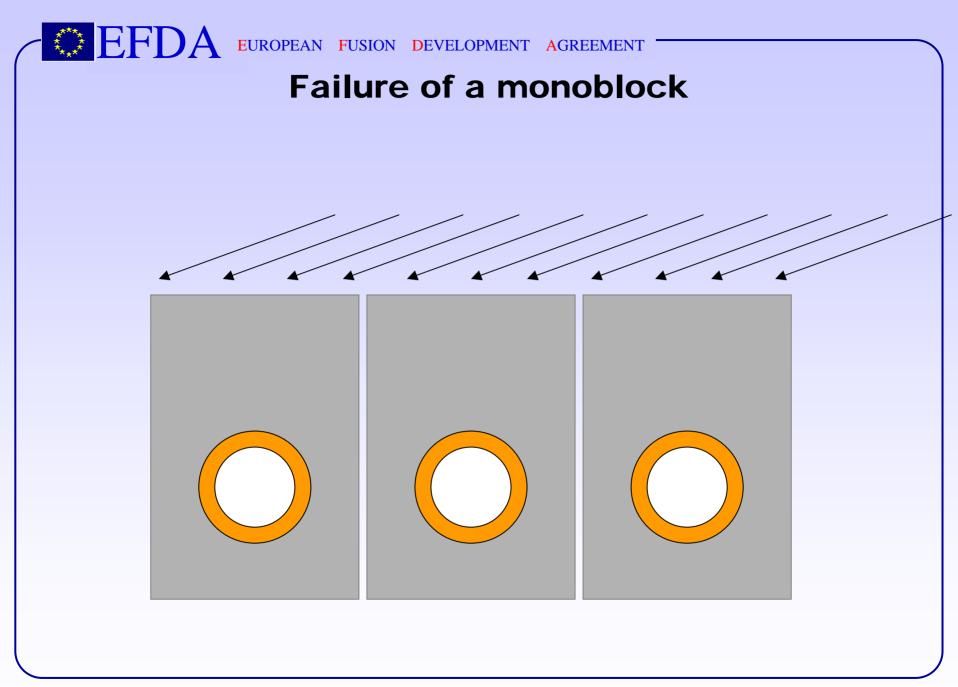
 To provide an analytical and experimental basis for the definition of acceptance criteria for the divertor PFCs

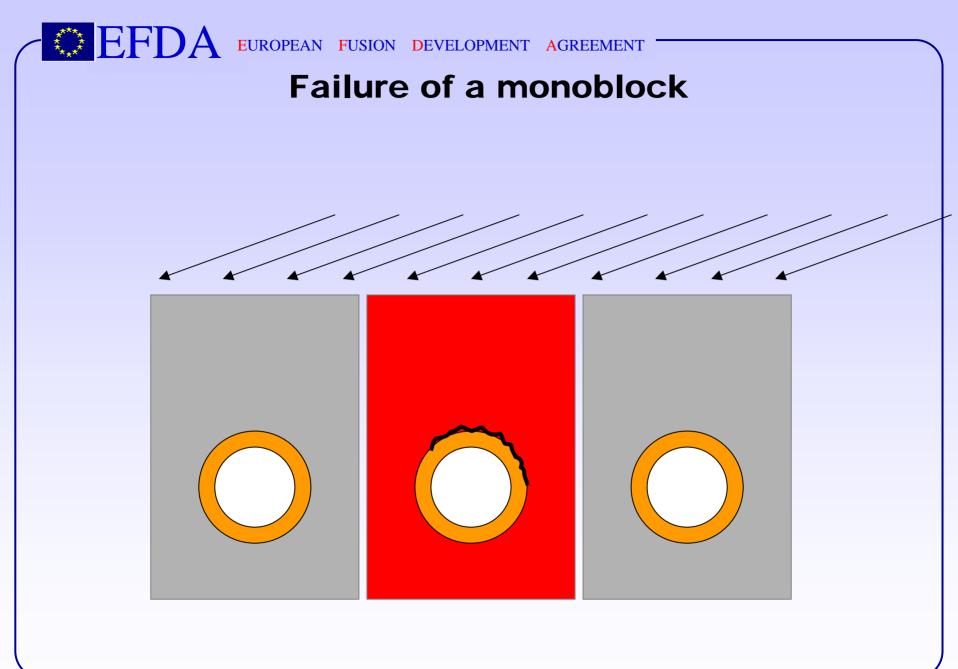
 To correlate this defect with the nondestructive testing evidence

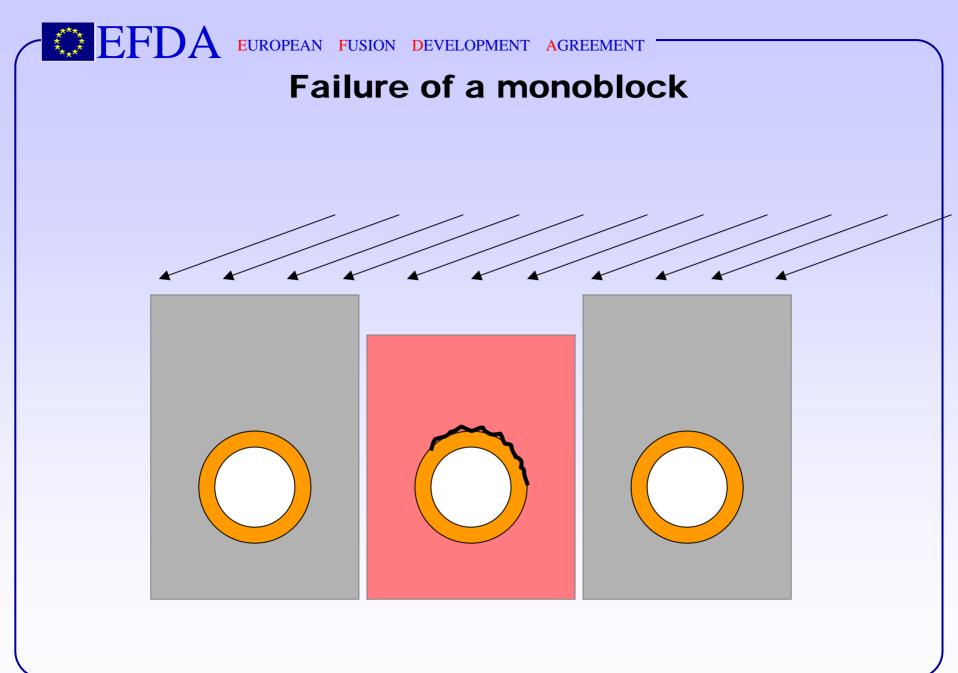










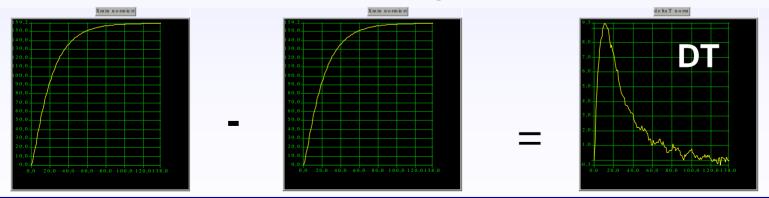


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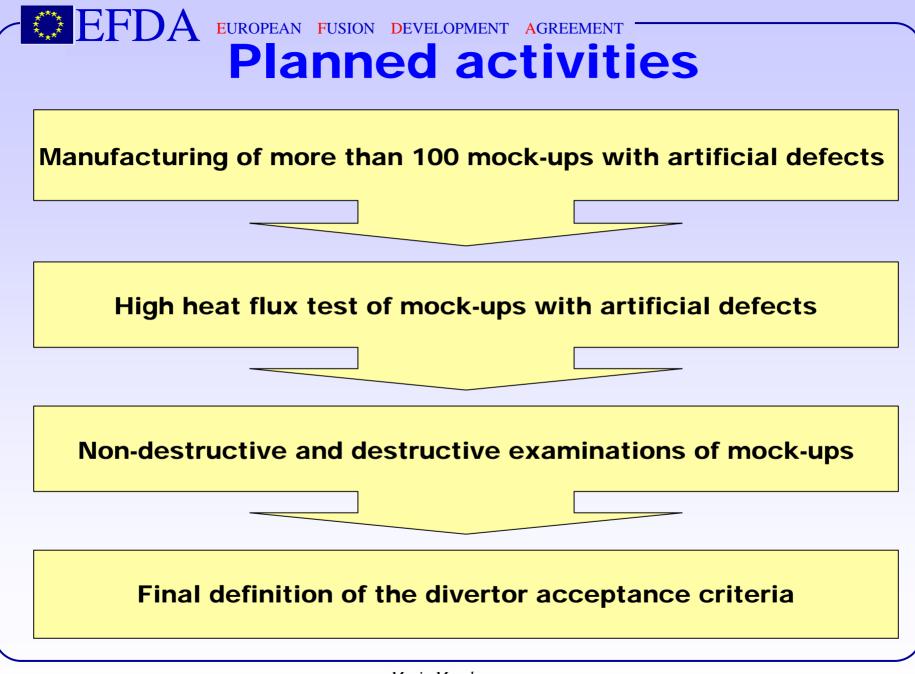
# Thermography Acceptance Criteria Statistical approach

In order to screen out defects that might lead to a CHF event or too high erosion the following <u>tentative</u> infrared acceptance values are proposed for discussion:

Less than 50% of the CFC monoblocks can have a DT > 4.0 °C Less than 5% of the CFC monoblocks can have a DT > 8.0 °C No CFC monoblocks shall be accepted with a DT > 10.0 °C



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### Present and future plans in the divertor area

- Completion of the on-going activities
- Optimisation of the existing HHF technologies and CFC materials
- Promoting competitions among industries
- Development of repairing methods
- Design supporting analysis
- Study of the effects of ELMs
- Diagnostic integration

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NDT methods during ITER procurement