

EUROMAT 2005,

5-8 September, Prague, Czech Republic



Modelling oriented experiments: Behaviour of Fe-Cr-C model alloys under neutron irradiation

Work supported by EFDA/TTMS-007

*Milena Matijasevic , Enrico Lucon
and Abdou Almazouzi*

Reactor Materials Research, SCK-CEN, Belgium

EUROMAT 2005,

5-8 September, Prague, Czech Republic



Content

- Introduction and Objectives
- Experimental techniques
- Results and discussion
- Conclusion



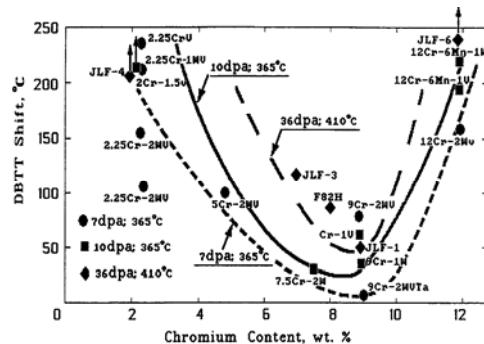
Introduction

In Fusion technologies:

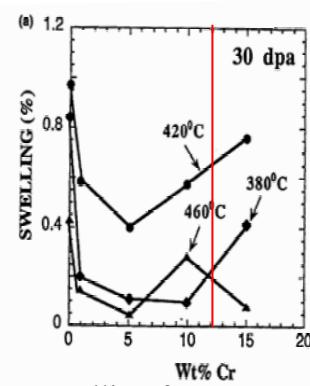
Operating conditions	Properties needed
Temperature (200 to 550°C)	High thermal conductivity and heat resistance; low thermal expansion
High flux and high dose (100 to 200 dpa); high He/dpa production rate (3 to 15 appm/dpa)	Low DBTT shift; sufficient strength with limited loss of ductility and toughness; low swelling rate
High stress level (100 MPa)	High creep resistance



Why 9% Cr ?



Lowest DBTT shift was found for 9% Cr steels → material is less brittle



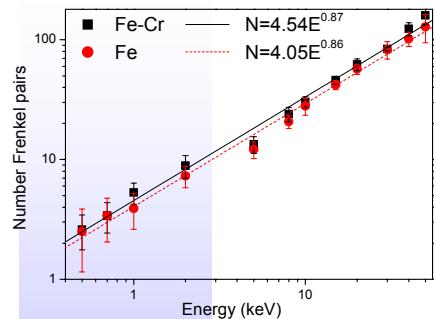
Low swelling for Cr-content between 5 and 10%

Fe-9%Cr seems to be the best candidate



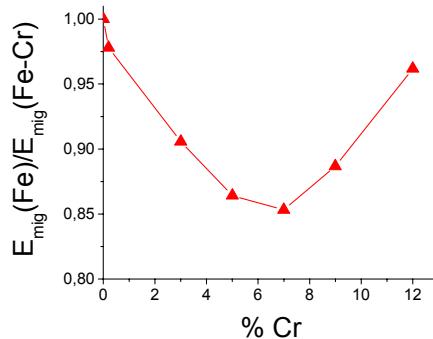
Effect of Cr on defect production and migration

Molecular Dynamic Simulation by D. Terentyev et al.



Cr has almost no effect on defect production rate

However: the presence of Cr affects strongly the diffusivities of irradiation induced defects



Objectives

- To assess experimentally the effect of Cr – concentration on defect production and accumulation in model alloys: how do they compare with steels under irradiation?
- To investigate the mechanisms of irradiation induced changes of the mechanical properties of high Cr F/M steels
- **Ultimate aim:** Provide reliable experimental database for model validation



Approach

- Experimental investigation of Fe, model alloys of different Cr content and industrial steels after neutron irradiation (same neutron flux, doses & temperature):

We chose as matrix:

- I- Pure Fe and ultra-pure Fe-9Cr
- II- Industrial pure Fe and Fe-2,5,9,12 Cr
- III- Conventional and LA Ferritic martensitic steels



Investigated Materials with chemical composition and heat treatment

Alloy	Mn	Si	P	Si	Al	Ti	Cr	Ni	Cu	Nb	B	Mo	C	N	V	W
251	0.009	0.02	0.013	0.0020	0.003	0.004	2.5	0.044	0.005	/	/	/	0.008	0.0173	0.001	/
259	0.02	0.04	0.011	0.006	0.0033	0.0028	5.04	0.06	0.01	/	/	/	0.02	0.0344	0.001	/
252	0.03	0.09	0.012	0.00066	0.0069	0.0034	9	0.07	0.01	/	/	/	0.02	0.0353	0.002	/
253	0.03	0.11	0.05	0.006	0.003	0.0037	11	0.09	0.01	/	/	/	0.03	0.0397	0.002	/

steel	C	Cr	Mo	W	Nb	Ta	V	P	Mn	Ni	B	N	Si
T91	0.1	8.32	0.96	<0.01	0.06	—	0.24	0.02	0.43	0.24	<0.0005	0.03	0.32
E97	0.12	8.96	<0.001	1.1	<0.001	0.13	0.19	<0.005	0.43	0.007	<0.001	0.016	0.07

Normalisation:

for alloys : **1050 °C for 3h,**

after air cooling

for steels: **1040-1070°C for 1h**

Tempering:

for alloys : **730°C for 4 h 10min,**

after air cooling

for steels: **730-780°C for 1h**

Experimental characterisation techniques (before and after irradiation)

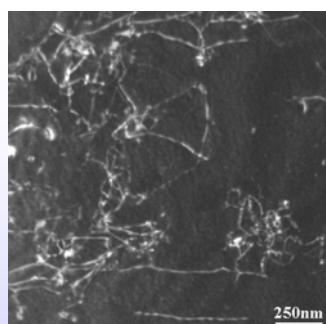
Microstructure:

- Metallography / Optical Microscopy
- SEM (Scanning Electron Microscopy)
- **TEM (Transmission Electron Microscopy) JOEL 3010:**
 - 1 mm disk specimen technique (magnetism)
 - Bright field (BF), Dark field (DF)
 - Weak beam dark field (WBDF), g(4g-6g)
- PAS

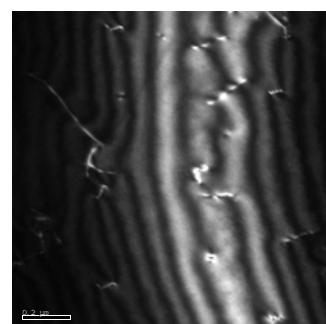
Mechanical properties:

- Hardness (HV5)
- **Tensile testing**
- Charpy
- Small punch
- Fracture toughness
- Compression (EPFL/CH)

Microstructure of pure Fe and ultra high pure Fe-9Cr before irradiation



Pure Fe

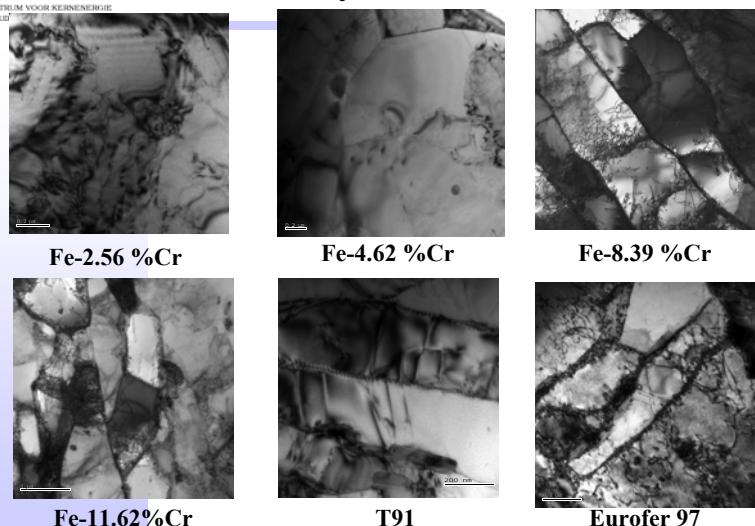


Pure Fe-9 Cr

- Ferrite , with grains size from 15-25 μm
- Dislocations are of type $b = a/2 <111>$, and their density is ($1.7 \times 10^9 \text{ cm}^{-2}$ for FeCr, and $7 \times 10^8 \text{ cm}^{-2}$ for Fe)



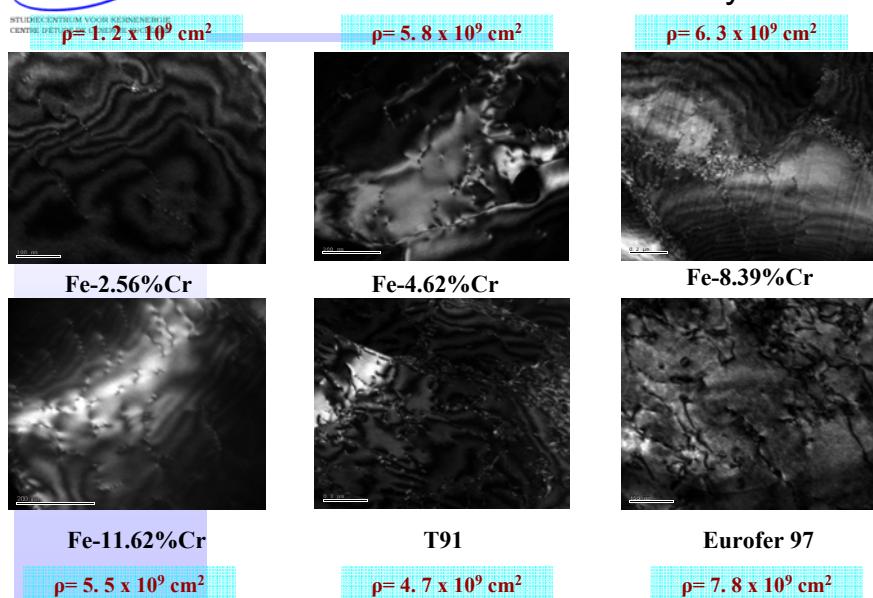
TEM micrographs with microstructure of Fe-Cr alloys and steels before irradiation



the structure changes from fully ferritic to ferrite + bainite: grain size decreases



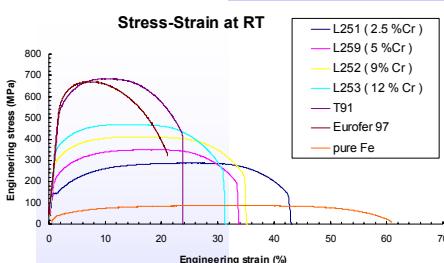
Dislocation structures in unirradiated material with their density



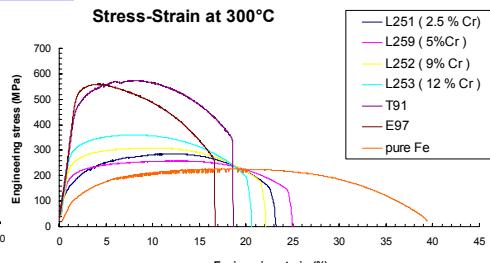
STUDECENTRUM VOOR KERNENERGIE
CENTRE D'ÉTUDE DE L'ÉNERGIE NUCLÉAIRE

Tensile tests

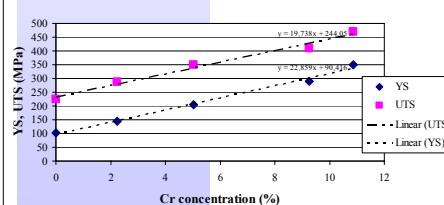
Stress-Strain at RT



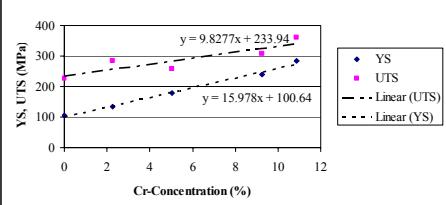
Stress-Strain at 300°C



Tensile properties at RT



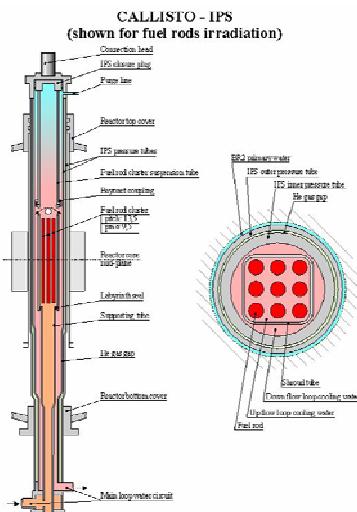
Tensile properties at 300°C

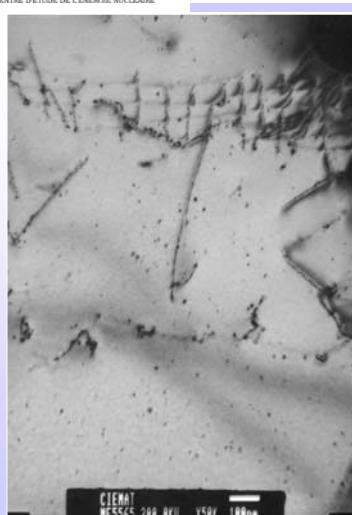
STUDECENTRUM VOOR KERNENERGIE
CENTRE D'ÉTUDE DE L'ÉNERGIE NUCLÉAIRE

Irradiation in BR2 (MIRE-Cr)

Irradiation conditions:

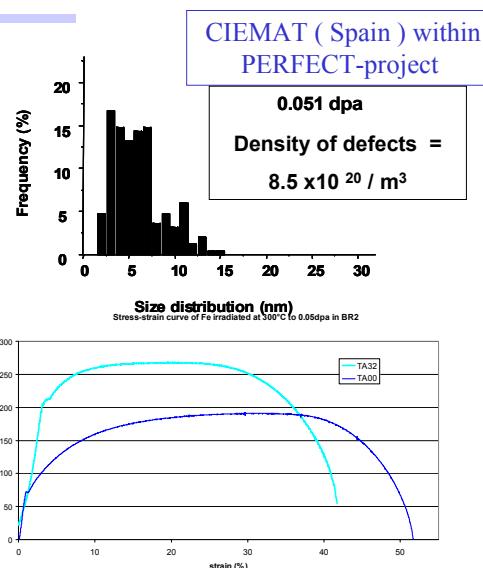
- Temperature: 300°C
- Pressure: 1.0 MPa ≤ P ≤ 15.7 MPa
- neutron flux ($> 1\text{MeV}$): $10^{13} \text{n}/(\text{cm}^2\text{s})$
- 3 groups of specimens for 1, 3 & 5 cycles
- 3 doses of 0.06, 0.6 and 1 dpa





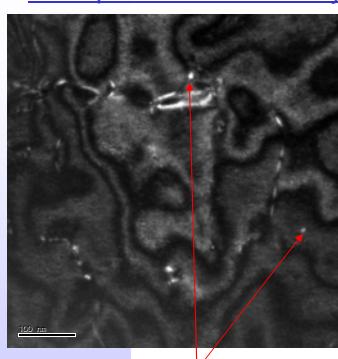
Pure Fe

Microstructure of pure Fe after irradiation at BR2

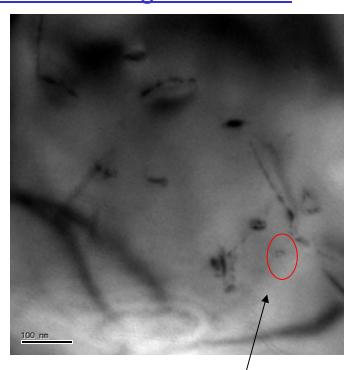


Defects induced by irradiation

- Need of WBDF (g, 4-6g) imaging technique:
[Example for 2.5 % Cr alloy with <110> image condition:](#)



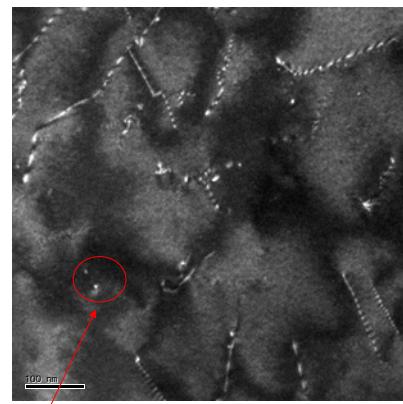
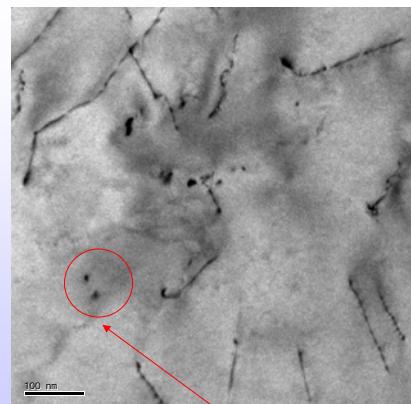
Defects
 $d = 6-8 \text{ nm}, \rho = 0.7 \times 10^{21} / \text{m}^3$



small loop
 $d = 12 \text{ nm}$



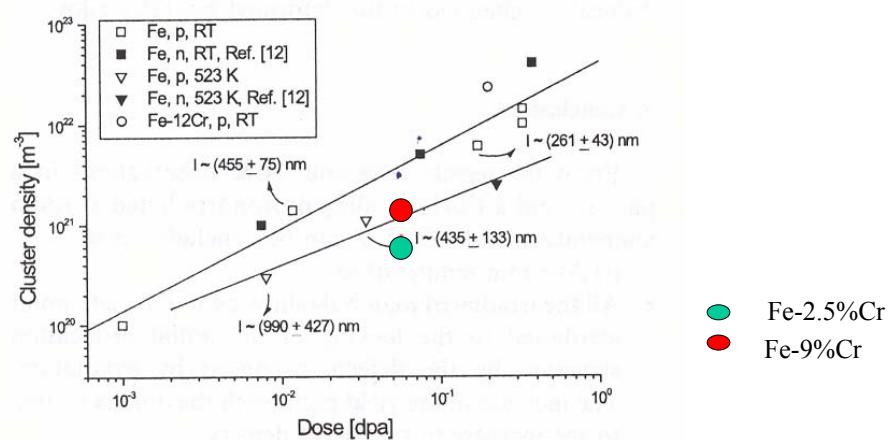
Fe-9% Cr alloy after irradiation



Defects , $d = 4 - 7 \text{ nm}$ and $\rho = 1.325 \times 10^{21} / \text{m}^3$



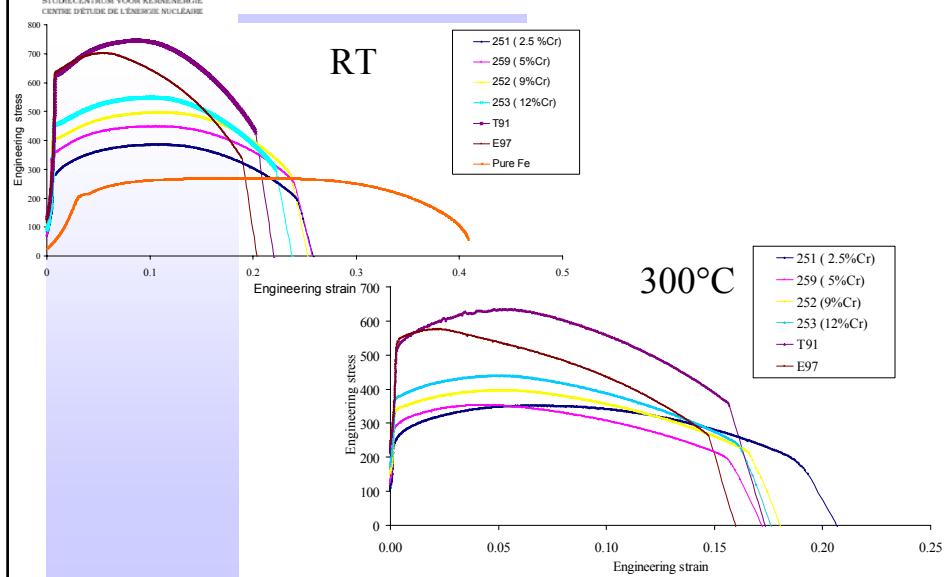
Dose dependence of the defect density in Fe and Fe-Cr alloys



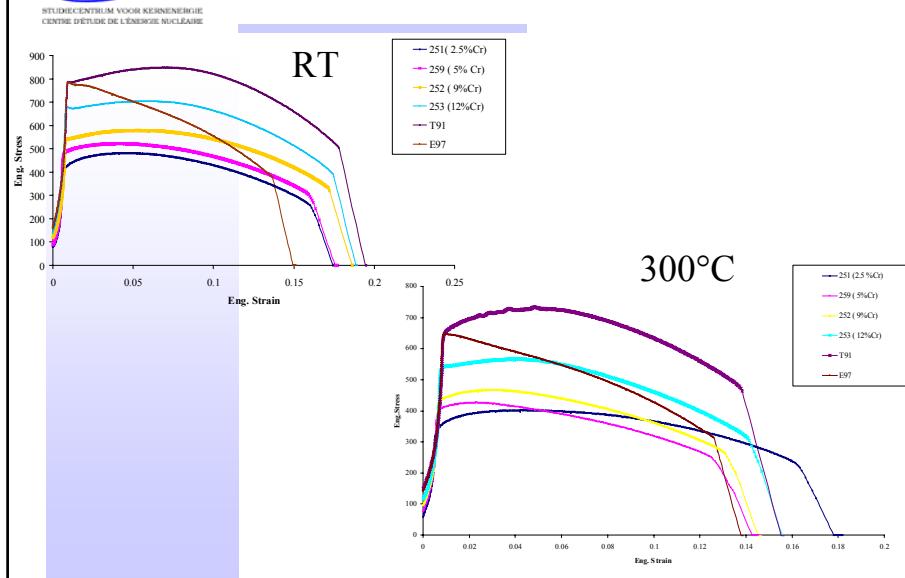
M.I.Luppo et al.JNM(2000)



Tensile test at RT and 300°C for 0.06 dpa



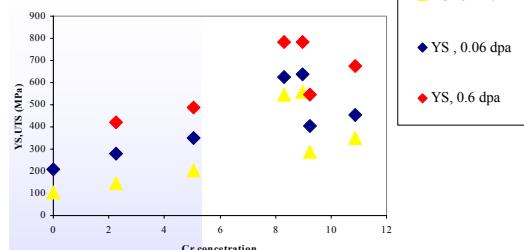
Tensile test at RT and 300°C for 0.6 dpa





Tensile properties and their comparison

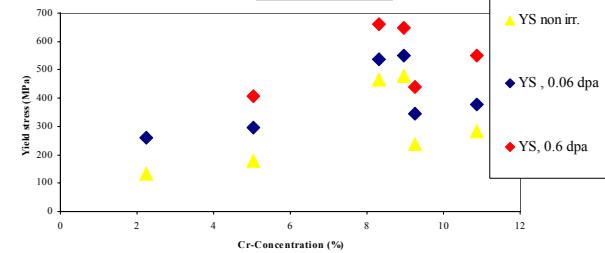
RT



Neutron irradiation at 300°C results in hardening of Fe-Cr alloys and it increases with doses

300 °C

Increase of YS can be observed, as well as loss of ductility , compared with non-irradiated material



Hardening

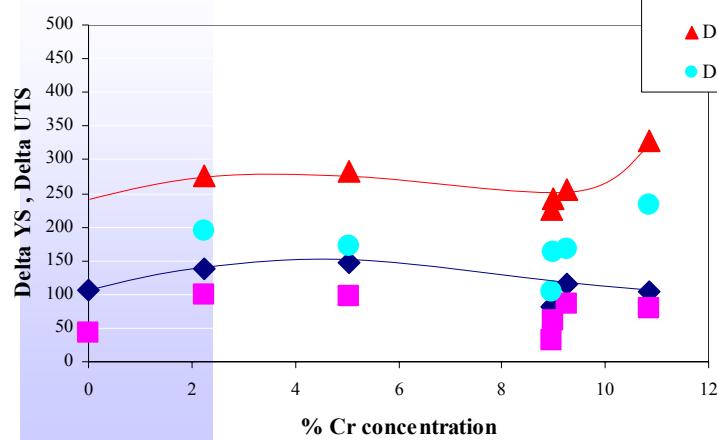
Irradiation induced hardening

Delta Yield 0.06 dpa

Delta UTS 0.06 dpa

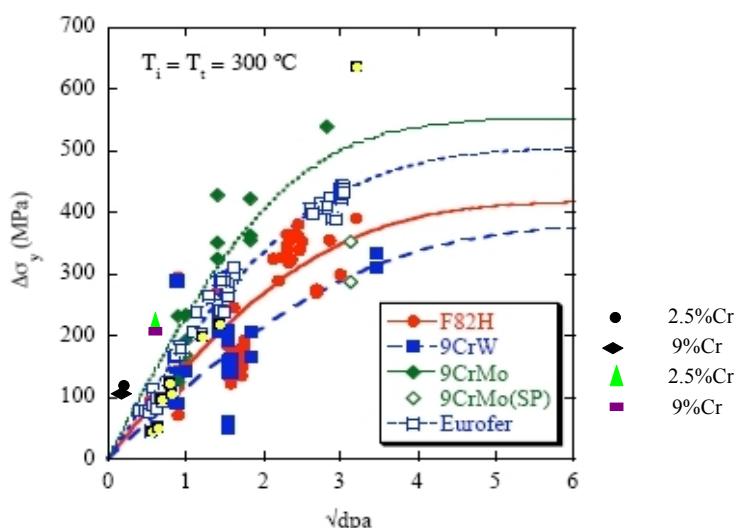
Delta Yield 0.6 dpa

Delta UTS 0.6 dpa





Comparison with other FM steels



Summary after irradiation at 300°C for doses of 0.06 dpa and 0.6 dpa

- Microstructure showed small defects induced by irradiation (**work is still in progress**)
- Neutron irradiation results in hardening of Fe-Cr alloys followed by reduction of their ductility
- The hardening depends on Cr concentration and it increases with doses of irradiation



Outlook

- The irradiation campaign should finish by end 2005
- 2005-2006 will hence be needed for PIE, using all mentioned techniques
- The combination of characterisation techniques on model materials will provide data on both microstructure and mechanical properties for systems close to steels
- These data will both be useful for model validation and to allow a wide correlation (model alloys, steels, ...) between microstructure, composition and mechanical behaviour under irradiation